

Thermal Ionization-Isotope Dilution Mass Spectrometry Determination of Ruthenium in Spent Nuclear Fuel[†]

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The optimum conditions for determining ruthenium in spent nuclear fuel were investigated by thermal ionization-isotope dilution mass spectrometry (TI-IDMS). Silica gel, boric acid and $Ba(NO_3)_2$ as an ionization enhancer and sucrose and HI solution as a reducing agent needed to be loaded with a sample on a rhenium filament. A single filament provided a large peak with increased stability and less isobaric effect than double filament. The filament current needed to be increased slowly to 5 A at a rate of 0.1 A/min to obtain good results. Distillation method was effective for separation of ruthenium from the isobaric elements in a sample solution. In this optimum analytical condition, ruthenium concentration of 1,570 µg/g was obtained from the spent nuclear fuel (average burn-up of 57,000 MWd/tU) sample.

Key Words: Ruthenium, Spent nuclear fuel, Determination, Thermal ionization - isotope dilution mass spectrometry.

INTRODUCTION

Ruthenium is a fission product generated when UO_2 fuel burns up in a nuclear reactor¹ and its isotopic ratio in a spent fuel can be used as an indicator of the irradiation behaviour of a nuclear fuel, similarly to U, Pu, Ku, Xe, Cs, Nd and Eu. Therefore, it is important to measure the isotopic ratio and the amount of ruthenium present in a spent fuel to assess the irradiation behaviour.

Several analytical approaches have been applied to the determination of ruthenium. When determining ruthenium spectrophotometrically², the oxidation state of the ruthenium in a sample should be adjusted precisely during the separation process. The detection limit, 5 µg/mL, is fairly high. Ruthenium can be determined with a detection limit of 0.005-1 µg/mL by atomic absorption spectrometer^{3,4} and inductively coupled plasma-atomic emission spectrometer⁵. However, these approaches are not good for highly radioactive samples because they require large amounts of sample. On the other hand, thermal ionization mass spectrometry (TIMS) is quite useful for analyzing very small amounts of highly radioactive samples. Gunji et al.6 studied thermal ionization - isotope dilution mass spectrometry (TI-IDMS) using a tungsten filament. 96Ru was used as a spike to determine the ruthenium in the spent fuel and the detection limit was found to be 0.005 μ g/mL. Huang *et al.*⁷ determined RuO₃⁻ by using negative thermal ionization-mass spectrometer (NTI-MS) because

ruthenium has high ionization potential. Devillers *et al.*⁸ and Poths *et al.*⁹ studied the enhancement of the ionization of ruthenium using silica gel and Ba(NO₃)₂.

In this study, TI-IDMS was applied to determine ruthenium in spent fuel and the analytical conditions were optimized using rhenium filament and ⁹⁹Ru spike. Ionization enhancers and reducing agents were added in the sample for determining ruthenium. A single filament was used and the filament current was increased slowly to 5 A to obtain good results. Distillation method was effective for separation of Ru from the isobaric elements and Ru in spent fuel was determined in this analytical condition

EXPERIMENTAL

TIMS (Finnigan MAT 262) with a variable multi-collector with nine Faraday cups and a secondary electron multiplier (SEM) detector was used for the simultaneous determination of isotopes. The mass resolution (m/ Δ m) is \geq 500 (10 % valley definition) and the abundance sensitivity for ²³⁸U is 1.0 × 10⁻⁶. A Milestone MLS-1200 MEGA microwave processor was used for the dissolution of ruthenium metal (⁹⁹Ru spike).

A 1000 μ g/mL ruthenium standard solution (Aldrich) was used for the experiment. A sucrose solution prepared by dissolving 1 g of sucrose (Merck) in 50 mL of distilled water was used as a reducing agent. HI and Ba(NO₃)₂ ICP-AES standard solutions were used as the reducing agent and ionization enhancer, respectively. A ⁹⁹Ru spike solution was prepared with

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ruthenium metal (Cambridge Isotope Laboratory, USA). To prepare the ⁹⁹Ru spike solution, ruthenium metal (⁹⁹Ru spike) was dissolved in a microwave processor. The ruthenium remaining in the vessel was filtered and heated with 500 mg of KOH in a nickel crucible in a furnace for 1 h at 300 °C, 1 h at 600 °C, 1.5 h at 700 °C and 0.5 h at 800 °C.

Separation of ruthenium from the sample solution: The distillation apparatus for the ruthenium separation is composed of a vessel in a heating mantle, a RuO_4 collector containing HCl and H_2O_2 solution and a NO_x gas collector containing diluted NaOH solution. The apparatus was set up in a hot cell. The distillation for ruthenium separation from a sample solution was carried out for 4 h at 90 °C.

The following simulated spent fuel solutions were used to test the distillation performance for ruthenium separation: (1) a ruthenium solution containing isobaric elements of Rh, Pd and Mo; (2) a solution containing the main fission products (Ru, Rh, Pd, Mo, Nd, Ce, La, Ba, Sr, Y, I, Cs and Zr); (3) a uranium solution containing the main fission products of (2).

A 0.555 g spent nuclear fuel sample (average burn-up of 57,000 MWd/tU; 4.5 % ²³⁵U; cooling time of approximately 4 years) was dissolved in a 1:1 nitric acid solution. The solution was then divided equally into three containers. Nothing was added to the first solution sample, 54 μ g of ⁹⁹Ru was spiked in the second solution sample and 100 μ g of Ru of the ICP-AES standard solution was added as natural Ru to the third solution sample. Because the natural isotopic abundance of Ru includes ⁹⁹Ru, which is not present in spent nuclear fuel, the experimental results using natural Ru and ⁹⁹Ru spikes were compared in this study.

RESULTS AND DISCUSSION

Re filament baking effect: The influence of the impurities in the unbaked rhenium filament was examined in the TIMS determination of ruthenium isotopes. The mass spectrum was recorded over the mass range (m/z = 84-104) of ruthenium isotopes after increasing the temperature of the rhenium filament without any sample; ⁸⁵Rb, ⁸⁷Rb, ⁹²Mo, ⁹⁴Mo, ⁹⁵Mo, ⁹⁶Mo, ⁹⁷Mo, ⁹⁸Mo and ¹⁰⁰Mo were detected. Rb and Mo are present as impurities in the rhenium filament. 96Mo, 98Mo and 100Mo affect the determination of 96Ru, 98Ru and 100Ru and the production of ${}^{85}\text{Rb}{}^{16}\text{O}{}^+$ (*m*/*z* = 101) by ${}^{85}\text{Rb}$ affects the determination of ¹⁰¹Ru isotope. Therefore, a filament was baked for 50 min at 5.5 A to eliminate the impurities. Next, the blank test was conducted again for the baked filament and its result showed smaller peaks than the unbaked filament. However, the Mo impurity still remained in the filament in low levels; thus, a correction was required for the isobaric Mo.

Ionization enhancers and reducing agents for Ru mass analysis: To examine the efficiency of TIMS for Ru isotope measurements, 3 μ L of Ru standard solution (for ICP-AES, 1000 μ g/mL) was analyzed. Several small peaks were observed at the beginning. However, they disappeared at increased temperature due to the volatility of RuO₄. Therefore, ionization enhancers, such as silica gel, boric acid and Ba(NO₃)₂, were loaded on a filament with Ru standard solution. However, the results were similar to the previous findings. On the other hand, when sucrose and HI solution were added to the Ru standard solution as reducing agents with ionization enhancers, the Ru isotope peaks were detected.

Stability of Ru isotope peaks according to the filament current: The single filament was more effective than the double filament for the isotopic ratio measurement of Ru by TIMS, providing a peak with sufficient intensity and a weaker isobaric effect (Fig. 1). The 101Ru measurement was repeated 20 times each at filament currents of 4.5 A, 5.0 A, 5.5 A and 6.0 A. Fig. 2 showed that the ¹⁰¹Ru peaks were stable at a filament current 4.5 A, but the intensity was quite low. The intensity of the peak exhibited similar trends at 5A, 5.5A and 6A. Therefore, the Ru measurement at a filament current of 5 A was used because the isobaric effect from the impurities contained in the filaments is lower than that for 5.5 A and 6 A. Additionally, the measurement of ¹⁰¹Ru was repeated for filament currents slowly increasing to 5 A at rates of 0.1 A/min, 0.2 A/min and 0.5 A/min. Stable large peaks were obtained when the filament current was increased at a rate of 0.1 A/min (Fig. 3).



Fig. 1. ¹⁰¹Ru peak intensity variation for repeated measurements with single and double rhenium filaments



Fig. 2. ¹⁰¹Ru peak intensity variation for repeated measurements with various rhenium filament currents

Ru separation through distillation: The solution mixed with Ru and its isobaric elements, such as Rh, Pd and Mo, was distilled to identify the separation efficiency of Ru from the other elements. The ruthenium solution collected through the distillation of the mixed solution was analyzed using ICP-AES. As shown in Table-1, only ruthenium and not any of the other elements, was detected. Consequently, ruthenium could be separated purely from the isobaric elements through distillation. To identify the recovery of ruthenium collected from spent fuel solution by distillation, ruthenium was separated from the simulated spent fuel solution and measured using



Fig. 3. ¹⁰¹Ru peak intensity variation for repeated measurements with various rhenium filament current intervals

ICP-AES. The ruthenium recoveries were 69, 71 and 74.2 % and the average value was 71.7 % (RSD, 3.1 %). Although the recovery was somewhat low, the distillation was regarded as a suitable method for the separation of Ru in the spent nuclear fuel, as the ruthenium determination was not affected by the low recovery when the spike was added to the sample solution before the distillation.

TABLE-1 ICP-AES RESULTS FOR THE SOLUTION SEPARATED FROM THE SIMULATED SPENT FUEL (μ g/mL) THROUGH DISTILLATION				
Samples	Ru	Rh	Pd	Мо
1	3.140	< 0.01	< 0.01	< 0.01
2	3.795	< 0.01	< 0.01	< 0.01
3	3.163	< 0.01	< 0.01	< 0.01
4	2.115	< 0.01	< 0.01	< 0.01
5	2.854	< 0.01	< 0.01	< 0.01
6	22.231	< 0.01	< 0.01	< 0.01

Ru determination of a spent nuclear fuel: The amount of Ru in the spent nuclear fuel was analyzed using methods proven with a simulated sample solution in this study. Three dissolved solutions of the spent fuel with an average burn-up of 57 GWd/tU were prepared. 99Ru spike, natural spike and no spike were added to three dissolved solutions and the ruthenium was separated by distillation. The mass spectrum for the Ru isotopes of the solution collected by distillation of the spent fuel dissolved solution (a) was obtained and compared to that of the standard solution (b) with natural Ru (Fig. 4). ⁹⁹Ru was detected in the standard solution but not the collected solution. Therefore, the ICP-AES standard solution with natural Ru could be used as a spike. The Ru isotopes were listed in Table-2 after the correction of the Mo isobaric effect using the value of ⁹⁴Mo/⁹⁹Ru. The amount of Ru in the dissolved solution of a spent fuel was analyzed by IDMS. As a result, 1570 µg/g Ru in the samples with ⁹⁹Ru spike and 1574 µg/g Ru with natural Ru spike (ICP-AES standard solution) were determined. These values were slightly smaller than the value (2000 μ g/g) obtained using the Origen code, likely due to low solubility of ruthenium in acidic solution.

Conclusion

The optimum conditions to determine ruthenium in a spent nuclear fuel by TI-IDMS were investigated. Silica gel, boric





Fig. 4. Mass spectra of Ru isotopes, (a): Ru solution collected by distillation of spent fuel; (b): ICP-AES Ru standard solution

TABLE-2 RU ISOTOPES CORRECTED WITH ⁹⁴ Mo/ ⁹⁹ Ru				
Isotope	Natural Ru (reference value)	Ru separated from spent fuel		
⁹⁶ Ru	0.1748	0.0165		
⁹⁸ Ru	0.0589	0.0259		
⁹⁹ Ru	0.4036	0.0092		
100 Ru	0.3991	0.2931		
101 Ru	0.5401	0.9568		
102 Ru	1.0000	1.0000		
¹⁰⁴ Ru	0.5911	0.6897		

acid, Ba(NO₃)₂, a sucrose and HI solution needed to be loaded with the sample on a rhenium filament. A single filament was used and the filament current was increased to 5 A at a rate of 0.1 A/min. Distillation method was effective for separation of Ru from the isobaric elements. The Ru concentration obtained from a spent fuel was approximately 1570 μ g/g. This value was slightly lower than the value of 2000 μ g/g obtained using the Origen code.

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